November 20, 2003

Mr. Lew W. Myers
Chief Operating Officer
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
5501 North State Route 2
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - REQUEST FOR ADDITIONAL INFORMATION RELATED TO LICENSE AMENDMENT REQUEST (TAC NO. MC0583)

Dear Mr. Myers:

By application dated August 25, 2003, FirstEnergy Nuclear Operating Company requested a license amendment to revise the technical specifications regarding the steam and feedwater rupture control system instrumentation setpoints and surveillance intervals. Based on the staff's review of your application, please provide additional information as discussed in the enclosure to this letter.

The enclosed request was discussed with Mr. D. Wuokko, et al., of your staff on October 30, 2003. A mutually agreeable target date of February 27, 2004, for your response was established. If circumstances result in the need to revise the target date, please contact me at (301) 415-2296 at the earliest opportunity.

Sincerely,

/RA/

Carl F. Lyon, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosure: Request for Additional Information

cc w/encl: See next page

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DATE	11/06/03	11/05/03	10/16/03	11/20/03

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The Honorable Dennis J. Kucinich, Member
United States House of Representatives
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REQUEST FOR ADDITIONAL INFORMATION

RELATED TO LICENSE AMENDMENT REQUEST DATED AUGUST 25, 2003

FIRSTENERGY NUCLEAR OPERATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

By letter dated August 25, 2003, FirstEnergy Nuclear Operating Company submitted a request to the Nuclear Regulatory Commission (NRC) to revise the Davis-Besse Nuclear Power Station, Unit 1 Technical Specifications (TS) to:

- Revise the Steam and Feedwater Rupture Control System (SFRCS) instrumentation TS to clearly identify the appropriate actions to be taken if an SFRCS instrumentation channel's output logic becomes inoperable,
- Relocate the SFRCS instrumentation trip setpoints from the TSs, and
- Decrease the channel functional test frequency from monthly to quarterly for the SFRCS instrument channels and make the associated changes to the trip setpoint allowable values.

Responses to the following request for additional information with regard to the Davis-Besse setpoint methodology and surveillance interval extension will allow the staff to complete its review in a timely manner.

Table 3.3-11, "Steam and Feedwater Rupture Control System Instrumentation"

1. The new action statement, Action 18, that is proposed in the amendment reads as follows:

   "With any component in the Output Logic inoperable, either declare the associated actuated component(s) inoperable, or place the associated actuated component(s) in the SFRCS-actuated position within one hour."

   Provide the technical basis for declaring the associated actuated components inoperable as this will allow more time than allowed for placing the components in the SFRCS actuated position within 1 hour.

Table 3.3-12, "Steam and Feedwater Rupture Control System Trip Setpoints"

1. In Table 3.3-12, the allowable values for Functional Unit 1 (Steam Line Pressure-Low), Functional Unit 2 (Steam Generator Level-Low), and Functional Unit 3 (Steam Generator Feedwater Differential Pressure-High) are all revised due to updated calculations and current setpoint methodology as stated in the amendment. Has the current setpoint methodology been approved by the NRC staff? If not, provide the setpoint methodology used to calculate the revised allowable values for the functional units mentioned above.

2. The amendment also states that the trip setpoints in the table are allowed to be removed since NUREG-1430 specifies only the allowable values for instrumentation functional units. To consistently reflect this specification, consider labeling the title of the table as:

   "Steam and Feedwater Rupture Control System Instrumentation Allowable Values"

Table 4.3-11, "Steam and Feedwater Rupture Control System Instrumentation Surveillance Requirements"

1. The methodology described in Electric Power Research Institute (EPRI) TR-103335-R1 is representative of extending calibration intervals based upon instrument drift analysis. The NRC staff has not accepted the EPRI TR and has issued a status report dated December 1, 1997, which documents the staff's issues with the report. The amendment requests surveillance interval extensions for channel functional tests from monthly to quarterly for the four Functional Unit 1 instrument channels on Table 4.3-11. The staff has previously accepted surveillance test interval extension requests based on a probability analysis result for core damage frequency and large early release frequency showing significantly lower increments together with a failure mode and effect analysis. Provide the technical basis for your request based on analysis that the staff has previously accepted.